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Criticality of Plutonium-Uranium Nitrate Solutions

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A series of experiments was performed providing new criticality data on plutonium-uranium nitrate solutions in cylindrical and spherical geometry. For the experiments in cylindrical geometry, the plutonium content of the total uranium plus plutonium was \sim 30 wt%; whereas, in the case of the water-reflected spheres, measurements were performed with both 15 and 30 wt% plutonium. The uranium in the mixture was slightly depleted, containing 0.66 wt% ²³⁵U. The plutonium concentration covered by these experiments ranged between 12.4 to 97.3 g Pu/L (uranium plus plutonium concentrations between 30 to 310 g/L). The ²⁴⁰Pu content of the plutonium was 5.6 wt% in the first case and 4.7 wt% in the second. The experiments were analyzed using ENDF/B-III cross-section data, and criticality factors were computed in each case. Some comparative calculations also were made, showing the differences obtained with ENDF/B-II, ENDF/B-III, and GAMTEC cross sections. The KENO code, with ENDF/B-III cross sections, as well as the HFN code, provide conservative results on the criticality factors for these systems. The average value of the computed keff for the cylinders, using KENO, was 1.022, and for the spheres, 1.024 using HFN. Thus, using these methods and cross-section data, the computed critical masses and volumes would be expected to be smaller than those measured by \sim 2% in terms of $k_{\rm eff}$.

INTRODUCTION

Criticality information is necessary for the design and operation of equipment and facilities for processing, fabrication, storage, and shipment of liquid-metal fast breeder reactor (LMFBR) fuel materials. Some experimental data were reported on criticality of homogeneous fuel materials containing both uranium and plutonium.¹⁻⁷ The objec-

tive of this series of experiments was to provide new criticality data on solutions in the 15 and 30% plutonium content range, adequate for use in assuring safe and economical criticality control throughout the LMFBR fuel recycle. The experimental data also provide useful benchmarks for data testing and analysis on integral criticality experiments for verification of the analytical techniques used in support of criticality calculations.

EXPERIMENT, DATA, AND ANALYSIS

Experiments were performed with plutonium-uranium nitrate solutions in water-reflected stainless-steel (Type 304L) cylindrical and spherical vessels. A schematic of the cylinder system is shown in Fig. 1. This vessel was 61.028-cm i.d. with a wall thickness of 0.079 cm, and was 107 cm in height. The water reflector surface was maintained at the top of the experimental vessel and was contained in a 102-cm-diam tank.

About 30 wt% of the plutonium-uranium of the nitrate solutions used in the cylindrical vessel was plutonium. The plutonium concentration varied from 12.4 to 97.3 g Pu/ℓ and the free nitric

¹J. H. CHALMERS, "Criticality Parameters for Mixtures of Plutonium Oxide, Uranium Oxide and Water," in *Criticality Control of Fissile Material*, pp. 3-11, International Atomic Energy Agency, Vienna (1966).

²S. R. BIERMAN and E. D. CLAYTON, *Trans. Am. Nucl. Soc.*, **15**, 307 (1972).

³L. E. HANSEN, S. R. BIERMAN, and E. D. CLAYTON, "Criticality of Mixed PuO₂-UO₂ Systems," in *Reactor Physics Quarterly Report*, BNWL-1150, pp. 5.6-5.16, Pacific Northwest Laboratories (1969).

⁴R. C. LLOYD and E. D. CLAYTON, *Trans. Am. Nucl. Soc.*, **17**, 269 (1973).

⁵S. R. BIERMAN and E. D. CLAYTON, *Trans. Am. Nucl. Soc.*, **15**, 805 (1972).

⁶R. C. LLOYD, E. D. CLAYTON, and S. R. BIERMAN, *Trans. Am. Nucl. Soc.*, **15**, 803 (1972).

⁷R. C. LLOYD, S. R. BIERMAN, and E. D. CLAYTON, *Nucl. Sci. Eng.*, **55**, 51 (1974).

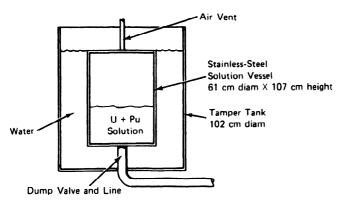


Fig. 1. Schematic of experimental set-up.

acid molarity changed from 1.3 to 4.8. The plutonium was in the tetravalent state. The 240 Pu content of the plutonium was 5.6%; the uranium contained 0.66% 235 U.

The critical approach method was used to obtain criticality, with the solution being pumped into the experimental vessel in incremental increases in height. The data obtained from the thirteen experiments performed are given in Table I. The critical heights were determined at various plutonium concentrations and acid molarities. The critical mass and volume are given for

each measurement. These results can be seen more easily in Fig. 2, where the mass and volume are given as a function of plutonium concentration. In this plot, the minimum mass of ~ 6 kg occurs at a concentration of ~ 70 g $(U+Pu)/\ell$. The volume curve near the minimum is very flat, but appears to reach a minimum at ~ 200 g $(U+Pu)/\ell$.

The criticality factor was computed in each case. The $k_{\rm eff}$ values included in Table I were computed using ENDF/B-III cross-section data in an 18-group calculation with the KENO Monte Carlo code. Epithermal energy groups were obtained via application of the ETOG (Ref. 9) code and the GAM portion of the EGGNIT code, 10 and the single-thermal group data via the FLANGE code 11 followed by the TEMPEST portion of the EGGNIT code. A thermal group energy cutoff

¹⁰C. R. RICHEY, "EGGNIT: A Multigroup Cross Section Code," BNWL-1203, Pacific Northwest Laboratories (1969).

TABLE I
Criticality of (U + Pu) Solution in Cylindrical Geometry

Plutonium Concentration ^{a,b}	Uranium		Cifi-	Critical	Critical	Critical Mass		k _{eff}
(g/l)	Concentration (g/l)	Н+	Specific Gravity	Height (cm)	Volume (£)	kg (Pu + U)	kg Pu	ENDF/B-III KENO
97.3 87.9 75.4 63.1 49.6 39.9 30.2 25.3 20.3 17.2	200.4 184.3 168.5 147.7 122.3 96.7 72.5 60.9 48.9 41.5	4.8 4.4 3.7 2.9 2.4 2.2 2.1 2.0 1.8 1.7	1.562 1.512 1.450 1.389 1.320 1.257 1.213 1.182 1.154 1.132	19.96 19.48 18.82 18.72 19.35 20.32 22.89 25.12 29.92 36.04	58.37 57.00 55.04 54.76 56.59 59.44 66.97 73.48 87.53 105.44	17.37 15.52 13.42 11.54 9.73 8.12 6.88 6.33 6.06 6.19	5.68 5.01 4.15 3.46 2.81 2.37 2.02 1.85 1.78	1.033 ± 0.008 1.035 ± 0.008 1.007 ± 0.008 1.026 ± 0.008 1.027 ± 0.008 1.016 ± 0.008 1.026 ± 0.008 1.019 ± 0.008 1.020 ± 0.006 1.023 ± 0.007
14.1 13.3 12.4	34.4 32.2 29.9	1.5 1.4 1.3	1.117 1.110 1.103	52.60 64.01 95.20	153.91 187.24 278.50	7.46 8.52 11.78	2.17 2.49 3.45	1.027 ± 0.006 1.007 ± 0.005 1.022 ± 0.005
							avera	$ge k_{eff} = 1.022$

a Isotopic composition:

238 Pu – 93.964 wt%
240 Pu – 5.585 wt%
241 Pu – 0.352 wt%
242 Pu – 0.072 wt%
235 U – 0.05 wt%
234 U – 0.05 wt%

242Pu-0.072 wt%
238Pu-0.027 wt%
238Pu-0.027 wt%

bChemical impurities:

$$\frac{Fe}{Pu} = 1.67 \times 10^{-2}$$
 by weight

$$\frac{Gd}{Pu} = 2.493 \times 10^{-4}$$
 by weight

⁸G. E. WHITESIDES and N. F. CROSS, "KENO-A Multigroup Monte Carlo Criticality Program," CTC-5, Oak Ridge Computer Technology Center (1969).

⁹D. E. KUSNER, R. A. DANNELS, and S. KELLMAN, "ETOG-1: A FORTRAN-IV Program to Process Data from the ENDF/B File to the MUFT, GAM, and ANISN Formats," WCAP-3845-1 (ENDF-114), Westinghouse Electric Corporation (1969).

¹¹H. C. HONECK and D. R. FINCH, "FLANGE-II (VERSION 71-1): A Code to Process Thermal Neutron Data from an ENDF/B Tape," DP-1278 (ENDF-152), Savannah River Laboratory (1971).

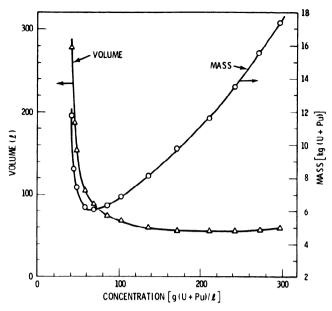


Fig. 2. Criticality of plutonium-uranium solution in cylindrical geometry.

of 0.683 eV was used in these calculations. Ten thousand neutron histories were used in the KENO calculations, resulting in a standard deviation of ±0.008. The computed criticality factors were found to be slightly above unity, with the average value of k_{eff} being 1.022.

The experiments in spherical geometry utilized low-240 Pu-content (~4.7 240 Pu) material with the plutonium concentration in the uranium being ~30 wt% of the total uranium and plutonium in the first

Stainless-Steel Solution Vessel Sphere Reflector Tank 102 cm diam Dump Valve and Line

Fig. 3. Schematic of experimental set-up.

Criticality of (U + Pu) Solution in Spherical Geometry

kelf	HFN HFN	1.022 1.018 1.031
Aass	kg Pu	1.69 1.06 1.38
Critical Mass	kg (Pu + U) kg Pu	5.45 3.34 9.37
Critical	(J)	23.90 30.14 30.18
Critical Badius	(cm)	17.87 19.30 19.31
Specific	Gravity	1.429 1.215 1.491
Acid	Molarity	3.12 1.49 2.1
Gadolinium	(g/l)	0.051 0.025 0.005
Uraniuma Concentration (g/1)		157.1 75.7 264.9
Plutonium* Concentration (g/t)		70.93 35.05 45.6
Sphere Wall Thickness	(cm)	0.112 0.122 0.122
Approximate Percent	ا ء	30 30 15

tion, wt%:	Uranium	235U 0.66 236U 0.01 238U 99.32	
isocopic composition, wt%:	Plutonium	238 Pu 0.01 239 Pu 95.09 240 Pu 4.66 241 Du 0.23	77.0

Experiment	Measured R_c (cm)	Gadolinium (g/l)	k _{eff} a	Calculated R_c with no Gadolinium (cm)
001R	17.87	0.051	1.022	17.14
007	19.30	0.025	1.018	18.36
011	19.31	0.005	1.031	19.26

TABLE III

Effect of Gadolinium on the Critical Spherical Radius

two experiments and ~15 wt% in the latter one. Figure 3 shows a schematic of the experimental set-up. The spheres were water-reflected with 18 cm of water above the sphere. The vent was 5.7-cm-i.d. tubing with a 0.3-cm-thick wall. Spheres of 35.7 and 38.6 cm diam were used in the experiments. These diameters were determined from accurate volume measurement. The sphericity of the vessels was very good; the maximum variation in the measured diameter was 0.1 cm. Table II gives the experimental data and the calculated k_{eff} values attained with ENDF/B-III cross sections. The 18-group multigroup constants were obtained using the GAMTEC-II code. The k_{eff} 's were calculated using the HFN code. values also showed a code-positive bias with an average of ~2.4\% above unity.

Experiments performed in the system before the present set of measurements involved the use of gadolinium poison. Some of the gadolinium from the lines and tanks was picked up during transfer of these solutions, even though considerable rinsing was done before beginning the experiments. The effect of this gadolinium contaminant is shown in Table III. The critical radii were calculated assuming the same k_{eff} as calculated but removing the gadolinium. Although the nuclear densities of the gadolinium were very small, its effect on k_{eff} was appreciable. Calculating k_{eff} disregarding the gadolinum gave k_{eff} as 1.047, 1.045, and 1.032, respectively, It is very important to include the effect of small quantities of such contaminants with large neutron cross sec-

In the case of the spheres, comparative calculations also were made showing the differences obtained using GAMTEC, ENDF/B-II, and ENDF/B-III cross-section data. The values are given in Table IV. For these experiments, the average $k_{\rm eff}$ is ~1.3, 1.6, and 2.4% high, respectively, with GAMTEC, ENDF/B-II, and ENDF/B-III cross-section data.

The results of these calculational correlations indicate we were able to calculate $k_{\rm eff}$ for the

TABLE IV

Comparison of Cross-Section Sets

(keff from HFN Code)

Experiment Number	GAMTEC	ENDF/B-II	ENDF/B-III
001R	1.011	1.015	1.022
007	1.007	1.010	1.018
011	1.020	1.024	1.031
Average	1.013	1.016	1.024

critical experimental systems with the (U+Pu) solutions to within $\sim\!\!2\%$ on the conservative side of criticality. However, note that no improvement was obtained in using the ENDF/B-III cross-section data, which gave, in fact, the poorest agreement in the sphere experiments.

CONCLUSION

The experimental data cover a wide range of (U+Pu) concentrations (30 to 310 g/ ℓ) and bracket the minimum critical mass and minimum critical volume. Using these data as calculational benchmarks, the results can be extended to areas of specific interest or need.

The KENO code, with ENDF/B-III cross-section data, as well as the HFN code, provides conservative results on the criticality factors for these systems; that is, the calculated values of $k_{\rm eff}$ are greater than the experimental ones. The average value of the computed $k_{\rm eff}$ is 1.022 for the cylinders and 1.024 for the spheres. Thus, the calculated critical masses and volumes would generally be smaller than those measured (by ~2% in terms of $k_{\rm eff}$).

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^aAssuming no gadolinium, the k_{eff} values would be 1.047, 1.045, and 1.032, respectively.