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CRITICALITY OF PLUTONIUM NITRATE SOLUTIONS IN SLAB GEOMETRY

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A series of criticality experiments was performed on plutonium nitrate solutions in slab geometry. The solutions contained plutonium at concentrations ranging between 58 and 412 g Pu/ liter for material with three different isotopic contents: 4.6, 18.4, and 23.2 wt% ²⁴⁰Pu. Acid molarities varied from 1.6 to 5.0.

The experiments were performed with a variable thickness slab-type vessel of 42-in. height and width, whose thickness could be adjusted throughout a range of 3 to 9 in. The experimental vessel was used with and without a water reflector and also with a 1-in.-thick Plexiglas reflector.

The critical experiment data from the finite slabs were corrected to yield values of critical thicknesses for one-dimensional infinite slabs, i.e., slabs of finite thickness but of infinite height and width. Analytical corrections, based on experimental data, were subsequently used to correct the critical infinite slab thicknesses for materials extraneous to the plutonium solutions, such as the effect of the stainless-steel vessel walls and room return neutrons. The analysis provided values for clean one-dimensional assemblies that were then used as an integral check of calculational methods using cross sections from the ENDF/B-II data file. The computed values of k eff for these "clean assemblies" ranged between 0.988 and 1.040; the values increased somewhat with increasing concentration.

INTRODUCTION

As the volumes needed for storage and processing of fissile materials increase, slab geometry systems become of special interest, due to the large quantities that can be safely handled without the addition of poisons or materials that inhibit mixing. Despite the frequent use of slab storage tanks, however, criticality data for plutonium solutions in slab geometry are particularly lacking. The first experiments to be reported on plutonium nitrate solutions in slab geometry were performed in France.¹ The data presented herein provide information at different plutonium concentrations, ²⁴⁰Pu isotopic contents, and excess nitrate molarities, supplementary to the French results. Of the three simplest (one-dimensional) geometries—the sphere, infinite cylinder, and infinite slab—the fewest data exist on the latter geometry for establishing nuclear criticality safety specifications.

EXPERIMENTAL DESCRIPTION

The slab assembly (Fig. 1) used in these experiments was of rather unique design in that its thickness could be adjusted over a range of 3 to 9 in. This range of adjustments was made possible by stainless-steel bellows fabricated around the periphery of the tank. The slab thickness change was accomplished by means of adjustment screws, located at the corners of the vessel, between the opposite sides of the slab; slab thickness was measured by means of a dial caliper. The height and width of the tank was 42 in., based on the average of the bellows variation. The stainless-steel sides (0.062 in.) were reinforced with an egg-crate-type structure that was effective in maintaining the side position to within ~0.010 in. on filling the tank. The assembly and its parts were of Type-304-L stainless steel. A reflected assembly was achieved by placing plastic in the egg crate or by attaching gasketed side plates and filling with water. The assembly was positioned in a large hood for contamination control. Experiments were performed at room temperature (23 ± 1℃).

Plutonium nitrate solutions of various acid molarities and plutonium concentrations with 4.6,² 18.4,³ and 23.2 wt%²⁴⁰Pu were used in these measurements. Table I lists the isotopic content of plutonium in the various nitrate solutions.

For each condition of reflection on the assembly, several critical geometries were determined by changing the thickness of the slab and measuring the resultant critical height. Critical slab heights were determined by a least-squares extrapolation of volume/count rate versus height measurements, using the normal critical approach method.

MEASUREMENTS AND RESULTS

Experiments were performed using plutonium having three different isotopic compositions. These included plutonium nitrate solutions at 58 g Pu/liter, 4.6 wt% ²⁴⁰Pu, with acid molarities of 2.3 and 5.0; a range of concentrations between 66 and 412 g Pu/liter, 18.4 wt% ²⁴⁰Pu; and concentrations of 202 and 284 g Pu/liter, 23.2 wt% ²⁴⁰Pu.

Several critical approach experiments were performed with each solution to determine the critical height as a function of the slab thickness. Using these determinations of critical dimensions



Fig. 1. Experimental slab tank assembly.

and by assuming constancy of buckling and extrapolation length, "best fit" values (i.e., minimum variation) were determined for the buckling and extrapolation length for each reflector condition. Table II gives values obtained for the buckling and extrapolation lengths of the various solutions, as well as critical infinite slab thicknesses derived for those same systems by buckling conversion from the experimental data. Although the material bucklings given may not be exact, their use for buckling conversion to infinite slab systems is well founded, due to the relatively large size of the experimental systems. The differences in critical thicknesses between a 42-in. slab and one of infinite dimensions are in the range of ~0.4 to

TABLE I

Isotopic Content of Plutonium in Nitrate Solutions

	wt% of Isotope					
Isotope	Solution 1 (4.6 wt% ²⁴⁰ Pu)	Solution 2 ^a (18.4 wt% ²⁴⁰ Pu)	Solution 3 ^b (23.2 wt% ²⁴⁰ Pu)			
²³⁸ Pu	0.006	0.19	0.070			
²³⁹ Pu	95.059	75.92	71.821			
²⁴⁰ Pu	4.671	18.40	23.185			
²⁴¹ Pu	0.255	4.53	3.958			
²⁴² Pu	0.009	0.96	0.966			

 $^{a_{241}}$ Am/Pu weight ratio = 0.0028.

 b_{241} Am/Pu weight ratio = 0.0021.

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0.7 in. for the cases studied herein. Bucklings derived for the water-reflected systems are slightly greater (up to 8%) than the values derived for the same solutions when unreflected. The phenomenon is probably caused by the shift in the neutron energy spectrum associated with the water reflector. Figure 2 shows the variation in derived extrapolation lengths as a function of plutonium concentration for the 18.4 wt% ²⁴⁰Pu solutions. These data are smoothly varying and increase as the plutonium concentration increases, thereby tending to confirm the self-consistency of the derived extrapolation lengths.

For water-reflected systems the slab vessel also achieved criticality when only partially filled. Consequently, since the vessel wall and reflector heights were constant, the extrapolation length for the top surface of the solution was determined such as to minimize the difference in buckling within each set of experiments through leastsquares analysis.

The uncertainties in the critical slab thicknesses presented in Table II are estimated to be ± 0.11 cm for a 95% confidence level. The infinite slab thicknesses obtained from buckling conversions of the experimental data are relatively insensitive to uncertainties in extrapolation lengths, as these errors are largely self-compensating in the conversion. The uncertainty in buckling conversion which results from errors in slab thickness and solution height, as well as the ex-

²⁴⁰ Pu Content	Plutonium Concentration (g/liter)	Acid Molarity	Reflector	Material Buckling (m ⁻²)	Effective Extrapolation Length (cm)	Critical Thickness of Infinite Slab of Solution Plus System ^a (cm)	
4.6	58.0	2.3	Unreflected	182.3	4.3	14.7	
4.6	58.0	2.3	Water	186.4	6.4	10.2	
4.6	58.0	5.0	Unreflected	170.0	4.17	15.7	
4.6	58.0	5.0	1-in. Plexiglas	170.5	5.64	12.8	
4.6	58.0	5.0	Water	170.2	6.60	10.9	
18.4	66.5	2.4	Unreflected	147.8	4.20	17.4	
18.4	66.5	2.4	1-in. Plexiglas	148.4	5.68	14.4	
18.4	66.5	2.4	Water	151.4	6.49	12.6	
18.4	160.0	3.0	Unreflected	141.0	4.46	17.5	
18.4	160.0	3.0	1-in. Plexiglas	136.0	6.30	14.3	
18.4	160.0	3.0	Water	145.2	6.90	12.3	
18.4	240.8	4.1	Unreflected	124.9	4.59	18.9	
18.4	240.8	4.1	1-in. Plexiglas	122.3	6.46	15.5	
18.4	240.8	4.1	Water	134.9	7.19	12.7	
18.4	412.1	4.2	Water	114.6	7.69	14.0	
23.2	283.8	2.2	Water	119.2	7.67	13.5	
23.2	201.6	1.6	Unreflected	134.9	4.2	18.7	
23.2	201.6	1.6	1-in Plexiglas	135.1	5.8	15.4	
23.2	201.6	1.6	Water	137.0	7.05	12.8	

 TABLE II

 Measured Criticality Data for Plutonium Nitrate Solutions

^aInfinite thickness with vessel walls and supporting grid structure identical to that of experimental finite assembly.

trapolation length, is ± 0.09 . Independent of this is the uncertainty of chemical analysis. Individual uncertainties are as follows:

Slab thickness measurement	±0.01	cm
Determination of critical solu- tion height	±0.040	cm
Chemical analysis	±0.060	cm
Uncertainty in extrapolation length	±0.3	cm

The data presented in Table III contain additional uncertainties as a result of the corrections applied for the effects of tank walls, support structure, and room return. These corrections are summarized in Table IV. Considering the latter, the uncertainty in the critical thicknesses presented in Table III is ± 0.2 cm.

It is interesting and, from a criticality safety viewpoint, important to note that the critical slab thickness is less for the 202 g Pu/liter solution than for the 284 g Pu/liter solution. The plutonium in both cases contained 23.2 wt% ²⁴⁰Pu, and the

lower concentration was produced by water dilution of the 284 g Pu/liter solution. Hence, the nitrogen-to-plutonium atomic ratio was held constant.

THEORY-EXPERIMENT CORRELATIONS

The HFN multigroup diffusion theory code,⁴ previously used to predict with good accuracy the criticality of plutonium solutions in spheres,⁵ also was used to compute criticality for the bare and water-reflected slabs.

Calculations, normalized to experimental data, were used to determine the effect of the stainlesssteel tank walls, support structure, and room return neutrons on the measured critical slab thicknesses. The necessary corrections were made in the manner discussed in a previous paper.⁶ The magnitudes of the corrections applied to the experimental slab thicknesses are given in Table IV.

Table III gives the unreflected and water-reflected infinite slab thicknesses corrected for

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Description of Pu(NO ₃) ₄ Solution	H/Pu (atom ratio)	Total NO₃ (g/liter)	Type Reflector	Critical Thickness of Infinite Slab as Derived from Experiments (cm)	Computed Criticality Factor, Multigroup Diffusion Theory- HFN Code	Computed Critical Thickness (cm)
58 g Pu/liter, 4.6 wt% ²⁴⁰ Pu, 2.34 <i>M</i> HNO ₃	425	203	Unreflected Water	16.91 9.13	1.014 0.999	16.60 9.15
58 g Pu/liter, 4.6 wt% ²⁴⁰ Pu, 5.00 <i>M</i> HNO ₃	383	397	Unreflected Water	17.99 9.78	1.000 0.988	18.00 10.08
66.5 g Pu/liter, 18.4 wt% ²⁴⁰ Pu, 2.36 <i>M</i> HNO ₃	367	219	Unreflected Water	19.79 11.38	1.020 1.002	19.17 11.31
160 g Pu/liter, 18.4 wt% ²⁴⁰ Pu, 3.04 <i>M</i> HNO ₃	144	360	Unreflected Water	20.05 11.21	1.027 1.022	19.17 10.49
202 g Pu/liter, 23.2 wt% ²⁴⁰ Pu, 1.60 <i>M</i> HNO ₃	115	313	Unreflected Water	21.19 11.70	1.022 1.008	20.34 11.41
241 g Pu/liter, 18.4 wt% ²⁴⁰ Pu, 4.14 <i>M</i> HNO ₃	89	495	Unreflected Water	21.48 11.64	1.031 1.014	20.34 11.15
284 g Pu/liter, 23.2 wt% ²⁴⁰ Pu, 2.23 M HNO ₃	78	436	Unreflected Water	12.43	1.007	21.40 12.13
412 g Pu/liter, 18.4 wt% ²⁴⁰ Pu, 4.16 M HNO ₃	46	664	Unreflected Water	13.01	1.040	21.90 12.23

TABLE III

Criticality of Infinite Slabs of	f Plutonium Nitrate Solution
(Experimental results and calculations	s with ENDF/B-II cross-section data

materials extraneous to the Pu(NO₃)₄ solution and reactivities computed for these slabs. Computed critical slab thicknesses are also given. All calculations were performed using ENDF/B-II crosssection data averaged over 18 broad energy groups by the GAMTEC-II code.⁷ The ENDF/B cross sections were obtained from the ENDF/B-II point energy library data through use of the FLANGE-II code⁸ to process the thermal data, and the ETOG code⁹ to process the epithermal data for broad group averaging in GAMTEC-II. All of the isotopes of plutonium, including the small quantity of 241 Am, were accounted for in these calculations. The 241 Am results from the beta decay of the 13-h half-life ²⁴¹Pu. Under some circumstances, the buildup of ²⁴¹Am can cause significant changes in the value of k_{eff} . Calculations show that ²⁴¹Am can, by itself, be made critical with sufficient quantity in an essentially unmoderated state (71.4 kg for a steel-reflected metal sphere).

For the two high ²⁴⁰Pu content solutions used in these experiments, the computed effect of the ²⁴¹Am was found to be small, amounting to a decrease in k_{eff} of from ~0.1 to 0.3%. The effect is sensitive to both ²⁴¹Am concentration and the hydrogen/plutonium ratio of the solution. The effect of 18.4 wt%²⁴⁰Pu on critical slab

thickness has been demonstrated in Fig. 3 where



Fig. 2. Extrapolation length-slabtank-18.4%²⁴⁰Pu.

TABLE IV

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Pu(NO3)4 Solution Slab Tank Corrections	2.23 <i>M</i> HNO ₃ 284 g Pu/lite1 23.2% ²⁴⁰ Pu (cr	-0.512	1	-0.512	1	1
	1.60 <i>M</i> HNO ₃ 202 g Pu/liter 23.2% ²⁴⁰ Pu (cm)	-0.584	+0.359	-0.476	+0.941	+1.24
	4.16 <i>M</i> HNO ₃ 412.1 g Pu/liter 18.4% ²⁴⁰ Pu (cm)	-0.414	-	-0.549	-	ļ
	4.14 <i>M</i> HNO ₃ 240.8 g Pu/liter 18.4% ²⁴⁰ Pu (cm)	-0.530	+0.365	-0.498	+0.955	+1.23
	3.04 <i>M</i> HNO ₃ 160 g Pu/liter 18.4% ²⁴⁰ Pu (cm)	-0.611	+0.357	-0.451	+0.926	+1.23
	2.36 <i>M</i> HNO ₃ 66.5 g Pu/liter 18.4% ²⁴⁰ Pu (cm)	-0.793	+0.338	-0.375	+0.865	+1.15
	5.00 <i>M</i> HNO ₃ 58 g Pu/liter 4.6% ²⁴⁰ Pu (cm)	-0.757	+0.338	-0.360	+0.866	+1.09
	2.34 <i>M</i> HNO ₃ 58 g Pu/liter 4.6% ²⁴⁰ Pu (cm)	-0.732	+0.326	-0.335	+0.832	+1.05
		Tank walls, reflected	Tank walls, unreflected	Support structure, reflected	Support structure, unreflected	Room return (13%), unreflected



Fig. 3. Effect of ²⁴⁰Pu on critical slab thickness of Pu(NO₃)₄ solution.

computed critical thicknesses are given for various concentrations of plutonium in plutonium nitrate solutions. These values were calculated for systems with no extraneous material and for solutions having no free acid. It was previously pointed out that the measured critical slab thickness was smaller for a concentration of 202 than at 284 g Pu/liter. For the plutonium nitrate solutions, the critical slab thickness will exhibit a minimum with plutonium concentration: the concentration at which this minimum critical slab thickness occurs depends on both the nitrate concentration and the ²⁴⁰Pu content of the plutonium. In the concentration range near this minimum, 150 to 200 g Pu/liter, it is estimated that the critical slab thickness would be doubled if the plutonium contained 25 to 30 wt%²⁴⁰Pu.

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REFERENCES

1. C. CLOUET D'ORVAL, E. DEILGAT, M. HOUELLE, and P. LECORCHE, "Experimental Research in France on Criticality Problems," *Criticality Control of Fissile Materials*, p. 193, International Atomic Energy Agency, Vienna (1966).

2. R. C. LLOYD, E. D. CLAYTON, and L. E. HANSEN, "Criticality of Plutonium Nitrate Solutions in Slab Geometry," *Trans. Am. Nucl. Soc.*, **11**, 381 (1968).

3. R. C. LLOYD, E. D. CLAYTON, and L. E. HANSEN, "Criticality of Pu(NO₃)₄ Solution Slabs Having High ²⁴⁰Pu Content," *Trans. Am. Nucl. Soc.*, 12, 871 (1969).

4. J. R. LILLEY, "Computer Code HFN-Multigroup, Multiregion Neutron Diffusion Theory in One Space Dimension," HW-71545, General Electric Company (1961).

5. R. C. LLOYD, C. R. RICHEY, E. D. CLAYTON, and D. R. SKEEN, "Criticality Studies with Plutonium Solutions," *Nucl. Sci. Eng.*, **31**, 165 (1966).

6. L. E. HANSEN, E. D. CLAYTON, R. C. LLOYD, S. R. BIERMAN, and R. D. JOHNSON, "Critical Parameters of Plutonium Systems, Part I: Analysis of Experiments," *Nucl. Appl.*, 6, 371 (1969).

7. L. L. CARTER, C. R. RICHEY, and C. E. HUGHEY, "GAMTEC-II-A Code for Generating Consistent Multigroup Constants Utilized in Diffusion and Transport Theory Calculations," BNWL-35, Battelle-Northwest (1965).

8. H. C. HONECK and D. R. FINCH, "FLANGE-II (Version 71-1): A Code to Process Thermal Neutron Data from an ENDF/B Tape," DP-1278 (ENDF-152), Savannah River Laboratory (1971).

9. D. E. KUSNER, R. A. DANNELS, and S. KELLMAN, "ETOG-I: A FORTRAN-IV Program to Process Data from the ENDF/B File to the MUFT, GAM, and ANISN Formats," WCAP-3845-1 (ENDF-114), Westinghouse Electric Corporation (1969).